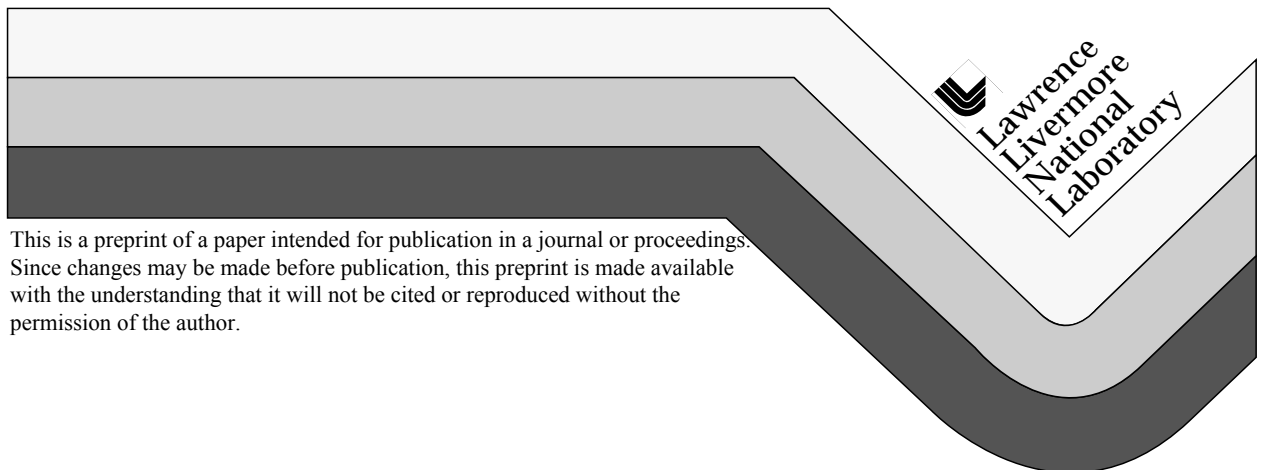


**SPATIALLY CONTINUOUS MIXED  $P_2$ - $P_1$  SOLUTIONS  
FOR PLANAR GEOMETRY**

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This paper was prepared for submittal to the  
ANS 2004 Winter Meeting  
November 14-18, 2004  
Washington, D.C.

June 2004



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# Spatially Continuous Mixed P<sub>2</sub>-P<sub>1</sub> Solutions for Planar Geometry

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## INTRODUCTION

Even-order Legendre polynomial (P<sub>N</sub>) expansion approximations of the neutron transport equation have historically seen only limited practical application. Research in the last decade [1] has resolved one of the historical theoretical objections [2] to the use of even-order P<sub>N</sub> approximations in planar geometry, namely the ambiguity in the prescription of boundary conditions as a result of an odd number of unknowns. This research also demonstrated the P<sub>2</sub> approximation to be more accurate than the P<sub>1</sub> approximation in planar geometry away from boundary layers and material interfaces. Neither the P<sub>1</sub> nor the P<sub>2</sub> approximation is convincingly more accurate near material interfaces. This progress motivated the reexamination of the multidimensional simplified P<sub>2</sub> (SP<sub>2</sub>) approximation [3], the development of P<sub>2</sub> approximations for planar geometry stochastic transport problems [4], and the examination of the P<sub>2</sub> and SP<sub>2</sub> approximations as a synthetic acceleration technique for the discrete ordinates equations. [5]

The major remaining objection to even-order P<sub>N</sub> approximations is that the scalar flux distributions obtained using these approximations can exhibit large spatial discontinuities at material interfaces and source discontinuities. In contrast, the odd-order P<sub>N</sub> approximations typically utilized give spatially continuous scalar flux distributions at these locations. In this paper, we propose a mixed P<sub>2</sub>-P<sub>1</sub> angular approximation designed to take advantage of the improved accuracy of the P<sub>2</sub> approximation in the interior of material regions and near external boundaries while retaining the continuous solutions obtained by the P<sub>1</sub> approximation near material interfaces and source discontinuities. We present numerical results from a series of eigenvalue calculations to demonstrate the accuracy of the mixed P<sub>2</sub>-P<sub>1</sub> angular approximation.

## THE MIXED P<sub>2</sub>-P<sub>1</sub> ANGULAR APPROXIMATION

The P<sub>2</sub> angular approximation to the planar geometry neutron transport equation in a spatial domain  $0 \leq x \leq L$  can be written as [1]:

$$-\frac{d}{dx} \frac{1}{3\sigma_{a1}(x)} \frac{d}{dx} \hat{\phi}(x) + \hat{\sigma}_{a0}(x) \hat{\phi}(x) = \hat{Q}(x) \quad , \quad (1)$$

where

$$\hat{\sigma}_{a0}(x) = \frac{\sigma_{a0}(x)}{1 + \frac{4}{5} \rho(x)} \quad , \quad (2)$$

$$\hat{Q}(x) = \frac{Q(x)}{1 + \frac{4}{5} \rho(x)} \quad , \quad (3)$$

$$\hat{\phi}(x) = \left[ 1 + \frac{4}{5} \rho(x) \right] \phi_0(x) - \frac{4}{5} \rho(x) \frac{Q(x)}{\sigma_{a0}(x)} \quad , \quad (4)$$

and

$$\rho(x) = \frac{\sigma_{a0}(x)}{\sigma_{a2}(x)} \quad . \quad (5)$$

Here  $Q(x)$  is a neutron source,  $\sigma_{an}(x) = \sigma_t(x) - \sigma_{sn}(x)$ ,  $n = 0, 1, 2$ , are the “absorption” cross sections obtained by subtracting the Legendre angular moments of the differential scattering cross section from the total cross section, and  $\phi_0(x)$  is the neutron scalar flux. Eq. (1) is a standard diffusion equation (with a modified absorption cross section and source) for the modified scalar flux unknown  $\hat{\phi}(x)$ . The neutron scalar flux,  $\phi_0(x)$ , is readily obtained from the unknown  $\hat{\phi}(x)$  using Eq. (4). Defining the parameter  $\rho(x)$  by Eq. (5) gives the P<sub>2</sub> approximation and setting  $\rho(x) = 0$  results in the standard P<sub>1</sub> approximation. Marshak vacuum boundary conditions for the P<sub>2</sub> approximation are given by [1]

$$\frac{1}{4} \left( \frac{1 + \frac{1}{2} \rho(x_b)}{1 + \frac{4}{5} \rho(x_b)} \right) \hat{\phi}(x_b) \mp \frac{1}{6\sigma_{a1}(x_b)} \frac{d}{dx} \hat{\phi}(x_b) = -\frac{3}{40} \frac{\rho(x_b)}{\sigma_{a0}(x_b)} \hat{Q}(x_b) \quad , \quad \begin{cases} x_b = 0 \\ x_b = L \end{cases} \quad (6)$$

In this paper, we propose to define the parameter  $\rho(x)$  by the equation

$$\rho(x) = \frac{\sigma_{a0}(x)}{\sigma_{a2}(x)} [1 - \exp(-p \sigma_t(x) d_i(x))] \quad , \quad (7)$$

where  $p$  is a user-specified parameter and  $d_i(x)$  is the distance to the nearest material interface or source discontinuity at a given spatial point  $x$ . For  $p = 0$ , the pure P<sub>1</sub> approximation is recovered for all  $x$ ; for  $p \rightarrow \infty$ , the pure P<sub>2</sub> approximation is obtained for all  $x$ . With this

representation for  $\rho(x)$  and  $1 \leq p < \infty$  (e.g.  $p \approx 5$ ), the  $P_1$  approximation is used at material interfaces and source discontinuities and the angular approximation transitions smoothly to the  $P_2$  approximation within approximately one mean free path from the material interface or source discontinuity. The numerical solution of the mixed  $P_2$ - $P_1$  equations requires essentially the same computational expense as the solution of the standard  $P_1$  equations.

## NUMERICAL RESULTS

In this section, we compare the accuracy of the mixed  $P_2$ - $P_1$  angular approximation to the pure  $P_1$  and  $P_2$  angular approximations for a series of two-region k-eigenvalue problems (numerical test problem IV.F of Ref. [1]). The test problem is a two-region, isotropically scattering critical slab 5.0 cm thick, with vacuum left and right boundaries and cross-section discontinuities at  $x = 2.5$  cm. The mathematical description of the transport problem is

$$\mu \frac{\partial}{\partial x} \psi(x, \mu) + \sigma_t \psi(x, \mu) = \frac{1}{2} \left[ \sigma_s(x) + \frac{\nu \sigma_f(x)}{k} \right] \int_{-1}^1 \psi(x, \mu') d\mu' , \quad 0 < x < 5 , \quad (8a)$$

$$\psi(0, \mu) = 0 , \quad 0 < \mu \leq 1 , \quad (8b)$$

$$\psi(5, \mu) = 0 , \quad -1 \leq \mu < 0 . \quad (8c)$$

The material properties are  $\sigma_t = 1.0 \text{ cm}^{-1}$  across the two regions of the slab,  $\sigma_s(x) = \sigma_s^l = \text{variable}$  in the left region,  $0 \leq x < 2.5$  (see Table I), and  $\sigma_s(x) = \sigma_s^r = 0.99 \text{ cm}^{-1}$  in the right region,  $2.5 < x \leq 5$ . The fission cross section  $\nu \sigma_f(x)$  takes the values  $\nu \sigma_f^l = 0$  in the left region and  $\nu \sigma_f^r = \nu \sigma_f^{\text{critical}}$  in the right region of the slab.

The critical values  $\nu \sigma_f^{\text{critical}}$  for each value of  $\sigma_s^l$  are given in Table I (taken from Ref. [1]) and give  $S_{16}$  discrete ordinates transport values of  $k = 1$ .

The percent relative errors in the computed  $k$  eigenvalue obtained using the pure  $P_1$  and  $P_2$  angular approximations to Eqs. (8) are shown in Table I. The accuracy of both approximations degrades as  $\sigma_s^l$  decreases, i.e. as the left region becomes less diffusive and the magnitude of the material property discontinuity increases. However, the eigenvalues obtained using the  $P_2$  approximation are significantly more accurate than those obtained using the  $P_1$  approximation for all values of  $\sigma_s^l$ . The scalar flux distributions surrounding the material discontinuity are shown in Fig. 1. The  $P_1$  scalar flux is continuous at the material interface but is significantly in error both near the interface and away

from the interface. The  $P_2$  scalar flux is discontinuous at the material interface but is more accurate than the  $P_1$  solution away from the interface.

The transport problem was also solved using the mixed  $P_2$ - $P_1$  approximation with three values of the parameter  $p$  of Eq. (7),  $p = 2.5, 5$ , and  $10$ . The scalar flux distributions near the material interface obtained using the mixed  $P_2$ - $P_1$  approximation are shown in Fig. 1. Larger values of  $p$  give scalar flux distributions with much of the character of the  $P_2$  solution except they are spatially continuous at the material interface. Smaller values of  $p$  give more diffusive scalar flux solutions.

The percent relative errors for the  $k$  eigenvalues obtained using the mixed  $P_2$ - $P_1$  approximation are shown in Table I. The mixed  $P_2$ - $P_1$  approximation is more accurate than the  $P_1$  approximation for all values of the parameter  $p$  and  $\sigma_s^l$  considered. For this test problem, smaller values of the parameter  $p$  give more accurate results for the  $k$  eigenvalue than larger values. With the exception of the  $\sigma_s^l = 0.99$  case, the mixed  $P_2$ - $P_1$  approximation is more accurate than both the  $P_1$  and the  $P_2$  approximations for all values of the parameter  $p$ .

## CONCLUSIONS

In this paper, we showed that the  $P_2$  approximation could be modified to give spatially continuous scalar flux solutions at material interfaces and source discontinuities. The proposed modification uses the  $P_1$  angular approximation at material interfaces and source discontinuities and smoothly transitions to the  $P_2$  approximation within approximately a mean free path from the interface. Neither the  $P_1$  nor the  $P_2$  approximation is convincingly more accurate near material interfaces, so the  $P_1$  approximation can be utilized at these locations to give spatially continuous results. Numerical results from a series of k-eigenvalue test problems demonstrate that the mixed  $P_2$ - $P_1$  approximation can yield improvements in accuracy over both the pure  $P_1$  and  $P_2$  approximations employed alone.

## ACKNOWLEDGMENTS

This work was performed under the auspices of the U.S. Department of Energy by University of California, Lawrence Livermore National Laboratory under Contract W-7405-Eng-48.

## REFERENCES

1. R. P. Rulko, E. W. Larsen, "Variational Derivation and Numerical Analysis of  $P_2$  Theory in Planar Geometry," *Nucl. Sci. Engr.*, **114**, 271 (1993).

2. B. Davison, *Neutron Transport Theory*, Oxford University Press, London (1957).
3. D. I. Tomašević, E. W. Larsen, "The Simplified  $P_2$  Approximation," *Nucl. Sci. Engr.*, **122**, 309 (1996).
4. B. Su, G. C. Pomraning, " $P_1$ ,  $P_2$ , and Asymptotic Approximations for Stochastic Transport," *Nucl. Sci. Engr.*, **120**, 75 (1995).
5. T. Noh, W. F. Miller, Jr., "The Effectiveness of  $P_2$  and Simplified  $P_2$  Synthetic Accelerations in the Solutions of Discrete Ordinates Transport Equations," *Nucl. Sci. Engr.*, **124**, 18 (1996).

TABLE I. Percent Relative Errors in the Criticality Eigenvalue  $k$ .

$\sigma_s^l$	$\nu\sigma_f^r = \nu\sigma_f^{critical}$	$P_1$	$P_2$	mixed $P_2$ - $P_1$		
				$p = 2.5$	$p = 5$	$p = 10$
0.99	0.1408299	-8.54	+0.19	-1.66	-0.70	-0.23
0.90	0.1650953	-9.86	+1.58	-0.68	+0.50	+1.07
0.80	0.1803974	-11.13	+2.57	-0.20	+1.21	+1.90
0.70	0.1903208	-12.09	+3.29	+0.06	+1.65	+2.46
0.60	0.1974249	-12.84	+3.86	+0.21	+1.95	+2.87
0.50	0.2028380	-13.45	+4.34	+0.30	+2.16	+3.19
0.40	0.2071421	-13.96	+4.76	+0.35	+2.33	+3.44
0.30	0.2106719	-14.40	+5.12	+0.38	+2.46	+3.66
0.20	0.2136351	-14.79	+5.45	+0.39	+2.56	+3.84
0.15	0.2149489	-14.96	+5.60	+0.40	+2.61	+3.92

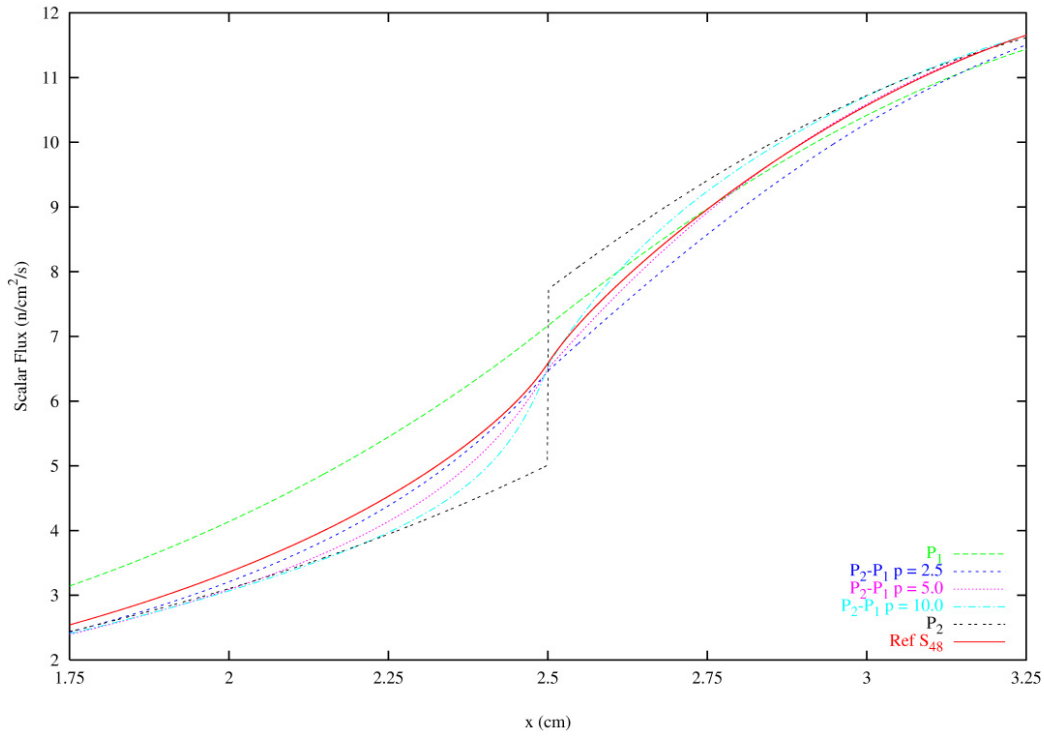


Fig. 1. Scalar flux near material property discontinuity for  $\sigma_s^l = 0.6$  case.

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